

Reference	Title	Authors	Year	Location	Document Type
	Dounreay Fast Reactor Metallurgical Problems	C. R. Tottle et al	1960	Proceedings of Symposium on the DFR, Paper 4, p55, IED Report 2	Report
	UKAEA 8 th Annual Report	UKAEA	1962	The Engineer, Volume 214, p48	
	Development and Performance of DFR Metal Fuel	S. A. Cottrell et al	1965	Proceedings of International Conference on Peaceful Uses of Atomic Energy, Geneva, Volume 10, p151	Conference
	DFR Fuel Element Behaviour up to the End of Reactor Period No 53	E. Edmonds, R. R. Higginson		DFR Fuel Element Behaviour up to the End of Reactor Period No 53	
	Limitations on the Performance of Reactor Fuels	Howe and Weber	2012	TID	
RMRC(66)35	DFR Fuel Element behaviour at High Burn-up	E. Edmonds, R. R. Higginson			
TRG Report 1296 (D)	Modifications to the DFR Outer Breeder	J. L. Philips			Report
TRG Memo 5351 (D)	Evidence of Changing Reactor Operating Conditions from DFR Driver Charge Fuel Element Examination	E. Edmonds, R. R. Higginson, B. E. Clark			Memo
FRDC/P1045	Comparative Surveys of Some Fast Reactor Fuels	J. E. Mann			
FRDC/P1061	Additional Surveys of Fast Reactor Fuels	J. E. Mann			
FRSC/FEWP/P(62)29	PFR Fuel: A Statement of Some Relevant Factors	J. F. W. Bishop			
	Proceedings of 2 nd International Conference on Peaceful Uses of Atomic Energy	O. J. Wick, T. C. Nelson, M. D. Freshley	1958	Geneva, Volume 6, p1776	
IQR-TM/C 557	The Compatibility of Thorium with Some Reactor Materials	M. D. Baines, M. Davis			
TRG Memo 620(D)		B. E. Hart, P. W. Jackson			Memo
	3 rd International Conference on the Peaceful Uses of Atomic Energy	B. R. T. Frost	1964	Volume 10, p153	
AERE-R-4324		H. Lloyd	1963		
AERE-R-4670		R. Paris	1964		
TRG Memo 1392 (D)		S. A. Cottrell, E. Edmonds	1962		Memo
FRSC/FEWP/P(62)49		S. A. Cottrell, E. Edmonds			
TRG Report 2789 (W)	Carbide Core Fuel For Fast Reactors	J. M. Horspool, G. P. Snape	1976		Report
TRG Report 1911 (R)			1966		Report
IAEA-SM-173/10	Fuel Element Behaviour Under Irradiation In DFR	K. Q. Bagley et al		Proceedings of Symposium on Fuel and Fuel Elements for Fast Reactors, Brussels, p87-100	Symposium
	UK Irradiation Experience Relevant to Advanced Carbide Fuel Concept for LMFBRs	K. Q. Bagley et al	1977	Topical Meeting Proceedings - Advanced LMFBR Fuels, Tucson, p313-325	Meeting
AERE - R 8462, FRDC/FEWP/P(76)41, FRDC/FRFF/P(76)13	Review of Liquid Metal Bonded Oxycarbide Fuel for Fast Reactors	R. G. Bellamy, R. Paris			
FRDC/FEWP/P(80)44	Review of Present Status of Bonded Carbide Fuel	R. Paris			
FRFF/(89)19	Experience Gained from Oxide Irradiations in Harwell MTRs	R. Paris			
	Mixed Oxide Fuel Performance	E. Edmonds et al	1979	Topical Meeting Proceedings - International Conference on Fast Breeder Reactor Fuel Performance, Monterey, p54 - 63	Meeting
FRSC/P(64)111	A Preliminary Specification of the Prototype Fast Reactor	A. G. Frame			
TRG Memo 4471(D), RMRC 28	Swelling in Fast Reactor Ceramic Pins	K. Q. Bagley, D. M. Donaldson	1967		Memo
FRSC/FEWP/P(68)449	The Influence of Fuel/Clad Interface Gap on Pin Performance	K. Q. Bagley			
FRDC/P(77)373	The Selection of Fuel Density for PFR	K. Q. Bagley, N. Parkinson			
FRDC/FEWP/P(79)41	Internal Clad Corrosion of highly Rated Oxide Fuel Pins Irradiated in DFR - Current Position	G. B. A. Linekar			
	Redistribution of Fuel and Fission Products in Irradiated Oxide Pins	J. I. Bramman, H. J. Powell	1975	Journal BNES, No1, p63-75	
	The Emission of Fission Products from Uranium/Plutonium Dioxide during Irradiation to High Burn-up		1970		
TP 1335	The Modelling of the Onset of Fuel Melting in Fast Reactor Fuel Pins	D. G. Martin, J. R. Matthews			
TP 1190	Some Experimental and Theoretical Studies of the Power to Initiate Melting in Various Fast Reactor Fuel Pin Designs	D. G. Martin, R. Paris, J. Needham			
AERE R 12702	The Effect of Rating Change on the Extent of Columnar Grains in Irradiated Mixed Oxide Fuel	C. B. A. Forty			
FRFF/P(89)29	Mixed Oxide Fuel Behaviour in DFR and PFR	R. Paris			
	The Irradiation Behaviour of Plutonium Bearing Ceramic Fuel Pins	H. Lawton, K. Q. Bagley, E. Edmonds	1966	Proceedings of Conference on Fast Breeder Reactors, BNES	Conference
	Fuel Clad Reactions in Irradiated Oxide Fuel Pins	W. Batey, K. Q. Bagley	1974	BNES Journal, Volume 49	
	Fuel/Cad Reactions in Mixed Oxide Fuel Pins	W. Batey, K. Q. Bagley			
	UK irradiation programme in support of Fast Reactor Development	J. O. Pounder et al	1973	ANS Topical meeting on irradiation experimentation in Fast Reactors, Wyoming, 23-34	Meeting
ND-R-37(W)		G. P. Snape			
	CRAMP: Core Restraint Analysis and Modeling Programme	R. C. Perrin	1979	Proceedings of 5 th SMIRT, Berlin	
	Use of PFR data for the validation of CRAMP	J. C. Duthie, R. G. Anderson, G. C. Critt	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 163-167	Conference
	Core management and fuel handling operations: experience on the UKAEA prototype fast reactor at Dounreay	A. M. Cruickshank	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 293-298	Conference
	Experience with the mechanical performance and distortion behaviour of the PFR absorber system	G. C. Crittenden, J. D.C Henderson	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 283-288	Conference
	Post irradiation examination experience of core distortion in PFR	P. R. Higginson, R. J. Lilley	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 307-314	Conference
05271/T4/014	The internal and external corrosion of clad in mixed oxide (U,Pu) oxide fuel pins	M. Mignanelli	1996	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	
	Effect of clad strain on fission product chemistry in Phenix pins at high burn up	M. Tourasse et al	1992	JNM Volume 188, 49-57	
05271/T3/001	Fuel column instability in PFR Annular Pelleted fuel pins	G. C. Crittenden	1997	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	
05271/T2/010	Data package for fuel pins from assembly RCHX	G. C. Crittenden	1996	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
05271/T2/034	Data package for fuel pins from assembly XCA	G. C. Crittenden	1997	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
05271/T2/011	Data package for fuel pins from assembly LJT	G. C. Crittenden	1996	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
05271/T2/018	Data package for fuel pins from assembly VGK	G. C. Crittenden	1996	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
05271/T2/006	Data package for fuel pins from assembly WSW	G. C. Crittenden	1995	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
	Experience with vibro compacted fuel irradiated in UK Fast Reactors	G. C. Crittenden, C. Brown	1992	Presentation to PNC, Tokai staff	Presentation
05271/T2/005	Data package for fuel pins from assembly DWS	G. C. Crittenden	1996	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
DEC/S3C/02	Post irradiation examination of PFR LVD and ANT pins at LECA/STAR	J. Noirot	2002	JNC/CEA/UKAEA/BNFL Agreement for the PIE of high burn pins from the PFR	
05271/T2/004	Data package for fuel pins from assembly DGS	G. C. Crittenden	1995	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
	Design of PFR fuel with larger diameter fuel pins	B. Walker, R. J. Lightowlers	1990	NES conference on Fast Reactor core and fuel structural behaviour, Inverness, 63-67	Conference
	Irradiation induced densification and Pu ₂ particle behaviour in mixed-oxide pellet fuel	M. D. Freshley, D. W. Brite, J. L. Daniel,	1979	JNM Volume 81, 63-92	
	Irradiation behaviour of mixed oxide driver fuel elements of Rapsodie-Fortissimo	F. Anselin, C. Allain, C. Mercier, G. Clott	1975	Trans ANS Vol. 20, p318-320	
	Evaluation of irradiation performance in Monju-type fuel assembly (MFA-1)	T. Donomae	1999	JNC TN9400 2000-075	
	FCMI in FBR fuel pins: observations and analysis	W. Dienst et al	1980	JNM Volume 91 73	
	Dimensional stability of FBR fuel pins with modified type 316 stainless steel cladding at high burn-up	H. Kashihara	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 243-147	Conference
	Transmutation of Minor Actinides: behaviour of Am and Np based fuels under irradiation	C. Prunier	1991	Paper 19.-1 International Conference on Fast Reactor and Related Fuel Cycles, Kyoto	Conference
	Irradiation performance of modified 316 stainless steel for Monju fuel	I. Shibahara, S. Ukal, S. Onose, S. Shik	1993	JNM, Volume 204 131-140	
	Status of liquid metal fast breeder reactor fuel development in Japan	M. Katsuragawa, H. Kashihara, M. Akebi	1993	JNM Volume 204, 14-22	
	Fuel to cladding gap evolution and its impact on thermal performance of high burn up fast reactor type uranium-plutonium O	I. Inoue, K. Maeda, K. Katsuyama, K. T	2004	JNM Volume 326, 59-73	
	Experience with failed LMR oxide fuel element performance in European fast reactors	H. Piltz, G. C. Crittenden, A. Languille	1993	JNM, Volume 204, 238-243	
	Operating experience with the Prototype Fast Reactor at Dounreay. International conference on Fast Reactors and related fue	C. V. Gregory	1991	Kyoto, 5.3-1 to 5.5-10	
05271/T3&5/001	Tabulation of PFR fuel pin failures	G. C. Crittenden	1995	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
	Grain boundary segregation in solution treated Nimonic PE16 during neutron irradiation	W. J. S. Yang	1982	JNM Volume 108 & 109, 339-346	
DMN/Dir 01-114	Reports on tensile tests performed on PE16 cladding from fuel pins irradiated in PFR assemblies VGK and LVD carried out by (CEA		2001		
SEMI/LCMI/RT/03-007/A	Reports on tensile tests performed on PE16 cladding from fuel pins irradiated in PFR assemblies VGK and LVD carried out by (CEA		2003		
05271/T2/033	Data package for fuel pins from assembly HSH	G. C. Crittenden	1997	Annular Pelleted fuel consultancy between UKAEA/BNFL and PNC	Consultancy
	Status of LMR fuel development in Europe	C. Brown, A. Languille, G. Muehling	1993	JNM Volume 204, 33-38	
	The Nuclear Fuel of Pressurized Water Reactor and Fast Reactors: design and behaviour	H. Bailly et al (editors)	1999	Chapter 6. ISBN 1-1898298-57-2	
	Status of fuel, blanket and absorber testing in the fast flux test facility	R. D. Baker et al	1993	JNM Volume 204, 109-118	
	UK studies of the performance of boron carbide control rod pins for the fast reactor	M. M. Oakden, B. Munro, J. E. Brocklehu	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 33-39	Conference
	The performance of boron carbide control rods in the Prototype Fast Reactor. Nuclear Energy	J. E. Brocklehurst, B. T. Kelly, K. E. Gilc	1984	Nuclear Energy, Volume 23, 179	
	Irradiation performance of Mk-II control rods in the experimental fast reactor JOYO	H. Kashihara, S. Shikakura, I. Shibahar	1990	BNES conference on Fast Reactor core and fuel structural behaviour, Inverness, 41 to 45	Conference
DFMC(89)P30	Thermal Hydraulic monitoring of subassemblies - results for Runs 16A, 16B and 16C	L. Martin			
AERE-R-3429	The Fabrication of Fuel Pellets from Mixed UO ₂ and PuO ₂ Powders	Russell, Brett, Harrison, Foster	1960		
TRG-R-692(D)	The Development of a Production Route for Mixed Oxide Fuel for the Prototype Fast Reactor	Adwick, Rose, Hartley	1964		
TRG-M-1849 (D)	The Effect of Heat Treatment on the Powder Characteristics of (U _{0.85} Pu _{0.15})O ₂	Adwick, Batchelor, Levack	1964		
TRG-M-2349(D)	The Theoretical Density of (U,Pu)O ₂	Adwick	1964		
TRG-R-808	Proceedings of the Colloquium on the Status of Mixed Oxide Fuel Development	Edited by Adwick	1964	DERE, 10-11	
	3 rd International Conference on Plutonium, 22-26 November 1965 - Influence of Powder Heat Treatment on Sintering of (UPu)O ₂	Adwick, Batchelor	1966	Institute of Metals	
FEWP/P(66)315	A Note on the Outgassing of Mixed Oxide Fuel in PFR	Hartley			
TRG-M-3592(D)	Fabrication of Sintered (UPu)O ₂	W. S. C. Reilly			
PG-M-1671 (W)	An Alternative Route for the Manufacture of a Vibrocompaction Feed for PFR Fuel Elements	B. T. Bell, D. Hope	1967		
WTC/TS/P216	The Properties of PuO ₂ relevant to its Long Term Storage	Ockenden, Gilbert	1971		
TRG-M-5869 (W)	The Fabrication of Uranium Dioxide Pellets of Controlled Oxygen-Metal Ratios and Density	G. R. Chilton	1972		
BNFL-M-291 (W)	The effect of Calcination Conditions on the Properties of Plutonium Dioxide and on the Quality of PuC ₂ -UO ₂ pellets prepared fr	Marshall, Holburt, Shoebottom	1973		
	Thermodynamic Aspects of the Use of Oxides as Nuclear Reactor Fuels	P. E. Potter, H. H. Rand	1980	High Temperature Science, Vol 13, pp 215 - 239	
ND-M-1964 (W)	FR Fuel Cycle Completion, Part I Fabrication of Mixed Oxide Pellets for the 0141FR-001 Sub-assembly	H. M. Macleod, G. R. Chilton, L. Watson,	1982		
ND-M-1894 (W)	PFR Fuel Cycle Completion, Part II Fabrication of Mixed Oxide Pellets for the 0141FR-001 Sub-assembly	Garrod, Kemp, Scroggings	1982		
ND-M-2791 (W)	PFR Fuel Cycle Completion, Part II Fabrication of Mixed Oxide Pellets for the 0141FR-001 Sub-assembly	L. C. Watson	1982		
FEWP/P(93)15	Problems associated with the use of Line 4 PuO ₂ to manufacture Fast Reactor Fuel	J. Edwards	1993		
ND-M-2444 (W)	Production of Mixed Oxide Fuel by the AERE Direct Pelleting Route for Gel-precipitated Spheres Part I - Preliminary Trials at V	H. M. MacLeod, L. C. Watson, G. R. Chill	1983		
ND-M-2446 (W)	Production of Mixed Oxide Fuel by the AERE Direct Pelleting Route for Gel-precipitated Spheres Part 2 - Kilogram Scale Trials	H. M. MacLeod, L. C. Watson, G. R. Chill	1984		
	Report on the Physical Blending of Line 4 PuO ₂ and UO ₂ powders	Bell	1984	AEA Internal Note	
ND-R-1055 (W)	Production Plant Experiments to improve the Shape and Solubility of PFR Fuel Pellets	J. Edwards, Lawrence, Smith	1984		
ND-R-1138 (W)	Some Studies on the Formation of Mis-shapen (U,Pu) ₂ O ₇ Pellets	J. Edwards, J. Drysdale, Lawrence	1985		
ND-M-3061 (W)	The Manufacture of Fuel at WNL for the First Speculative Advanced Design Sub-Assembly (Low O/M)	L. C. Watson, Smith	1985		

Reference	Title	Authors	Year	Location	Document Type
FFWG/P(86)6	The Effect of Calcination Temperature on the Properties of Repackaged PuO ₂	Robertson, Smith, Nuttall	1986		
ND-M-3918 (W)	A Comparative Assessment of Alternative Techniques for the Simultaneous Milling and Blending of Powders	H. M. MacLeod	1987		
FFWG/P(88)13	Development of a Spheroidiser for MOX Fuel Fabrication, Part 1 – Trials with Ceria	Thompson	1988		
FFWG/P(89)31	Flow-sheet Implications of Alternative Finishing Processes	Langley, Lyon, Wace	1989		
AEA-FS-0050 (W)	The Developments and Manufacture of MOX Fuels in the UK	Bell, J. Edwards, H. M. MacLeod	1989	MOX Conference Paper	
	A Review of Data on the Influence of Feed Powder Properties on the (U,Pu)O ₂ Fuel Pellet Manufacture	H. M. MacLeod	1991		
	The Determination of Density Distributions in Ceramic Compacts using Auto-Radiography	H. M. MacLeod, K. Marshall	1977	Powder Technology, Vol 16, p107	
	Oxygen Potentials of Uranium-Plutonium Oxides in the New Stoichiometric Region	J. Edwards, R. N. Wood, G. R. Chilton	1985	J Nuclear Materials, Vol 130, p 505	
	A Versatile Facility at Windscale, United Kingdom, for the Fabrication of Experimental Uranium-Plutonium Oxide Nuclear Fuel	H. M. MacLeod, G. R. Chilton, J. A. Dodc	1989	Nuclear Technology, Vol 88, p 190 – 200	
	Development of Mixed Oxide Fuel Manufacture in the United Kingdom and the Influence of Fuel Characteristics on Irradiation	H. M. MacLeod, G. Yates	1993	Nuclear Technology, Vol 102, p 3 – 17	
	UK Development Toward Remote Fabrication of Breeder Reactor Fuel	R. L. Nelson, N. Parkinson, W. C. L. Ken	1981	Nuclear Technology, Vol 53, pp 196 - 203	
	Manufacture of First Fuel Element Charge for PFR	A. S Davidson, T. L. J. Moulding	1973	Symposium, IAEA, Brussels	Symposium
	Sol-Gel and Gel Precipitation Processes – Status Report	C. J. Hardy	1968	IAEA, Panel on Sol-Gel Processes, Vienna	Panel
	Gel Process Development in the UK	C. H. Hardy, E. S. Lane	1970	Symposium, Gatlinburg, Tennessee	Symposium
	Gel Process Development in the UK – Status Report	H. A. Taylor	1973	IAEA, Panel on Sol-Gel Processes, Vienna	Panel
FSSG/P(88)6	Solubility of Commercial Fast Reactor Fuel	J. Edwards			
FRFF/P(89)23	uel Dissolution Studies in the UK – A Summary of Work Carried out in December 1988	J. Edwards			
ND-M-1031 (D)	he Kinetics and Mechanism of FR Fuel Dissolution Work, Post 1978	J. A. Crofts et al	1987		
	UK Fast Reactor Vibro Fuel Fabrication Experience	G. R. Chilton and H. M. MacLeod	2000	BNFL report prepared for JNC under terms of Agreement for the Provision of Data Relating to FR Vibro Fuel Fabrication and Perfor Report	
	UK Fast Reactor Vibro Fuel Fabrication Experience and Fuel Performance – Phase II	G. R. Chilton and J. M. Wright	2001	BNFL report prepared for JNC under terms of Agreement for the Provision of Data Relating to FR Vibro Fuel Fabrication and Perfor Report	
		C. Cawthorne, E. J. Fulton	1967	Nature 216, 515	
		Greenwood, Foreman and J. Rimmer	1959	J Nucl. Mat. Vol 1 (4), 305 – 324	
TRG Report 1535 (C)	Conclusions of the PFR Fuel Cladding Panel on the Choice of Cladding Material for the First Fuel Charge in PFF	Approved by J.M. Hutcheon	1967		Report
	Proc. Int. Conf. on Solid State Physics Research with Accelerators	G. Lewthwaite, D. Mosedale, Ward	1967	Nature 216, 472	
		R. V. Heskeith	1967	Brookhaven National Laboratory, USAEC Report BNL-50083, p 389	Report
		D. Mosedale, G. W. Lewthwaite, G. O. Li	1969	Nature 224, 1301	
FRDC/FEWP/P(74)869, DFMC/P(74)15	Irradiation Creep Data for fast Reactor Design and Modelling	D. Mosedale			
	Irradiation Creep in Non-Fissile Metals and Alloys	D. R. Harries	1977	J.Nucl.Mat.,Vol 65, 157 – 173	
CFWG/P(93)16	Irradiation Creep of Nimonic PE16 Pressurised Tubes irradiated in PFR at doses up to 90 dpa over the temperature range 420	J. S. Watkin, B. Munro, S. W. Adaway, A. Bancroft			
	Compilation of Materials Data Sheets with Design Rules for the Materials Properties of Nimonic PE16 for EFR Fuel Pin Design	Calculations - AGT1 - SG2-1			
TRG Memo 7160 (D)	Rules for Calculation the Voidage Swelling of Fast Reactor Materials – 5 th edition	J. K. Butler et al	1976		Memo
	Void Swelling Behaviour in UK Steels	C. Brown et al	1983	Proc. Conf on Dimensional Stability and Mechanical Behaviour of Irradiated Metals and Alloys, Brighton, Vol1, pp63 – 67	Conference
	Void Swelling in Austenitic Stainless Steels – a Review	J. I. Bramman	1983	Proc. Conf on Dimensional Stability and Mechanical Behaviour of Irradiated Metals and Alloys, Brighton, Vol 2, pp 35 – 35	Conference
DFMC/P(84)22	Cladding Strains in Irradiated PFR Fuel Pins	G. C. Crittenden et al			
DFMC/P(83)29	Some Comments on the Void Swelling of CW EN58B, ST&A PE16 and 20% CW M316 in PFR	G. A. B. Linekar			
	Void Swelling and Microstructural Features in the Cladding of PFR Pin 100001 Irradiated to 9% ha max burn-up in cluster RG	C. Brown et al	1981	PFR Cladding Panel Note 672	Panel
	The Void Swelling Behaviour of PFR Pin 102529 from Sub-assembly RST – max Burn-up 4.17%	C. Brown et al	1981	PFR Cladding Panel Note 673	Panel
	Density Changes in Cladding Materials Irradiated in DFR	J. I. Bramman et al	1971	Proc.of Conf. on Voids Formed by Irradiation of Reactor Materials, Reading, pp 27-33	Conference
	Void Swelling Studies of Austenitic Fuel Pin Cladding Material Irradiated in the Dounreay Fast Reactor	C. Brown et al	1979	Proc. Int.Conf. on Irradiation Behaviour of Metallic Materials for Fast Reactor Core Components, Ajaccio, pp129 – 136	Conference
	Void Swelling and Precipitation in a Neutron-Irradiated, Niobium-Stabilised Austenitic Stainless Steel	T. M. Williams et al	1982	J. Nucl. Mat. Vol 107, pp222-224	
DFR 445/1, ND memo 422(D)	An Investigation of Three 20%CW FV548 Pins Exhibiting High Local Clad Strains in Experiment	G. A. B. Linekar et al	1978		Memo
	Annealing Experiment on CW FV548	T. M. Williams	1978	PFR Cladding Panel Note 478	Panel
	Primary Recrystallisation of Cold Worked FV548 and M316 Austenitic Steels	D. R. Harries and T. M Williams	1980	PFR Cladding Panel Note 582	Panel
	Primary Recrystallisation and Softening Characteristics of modified CW FV548-type steels	R. M. Boothby et al	1981	PFR Cladding Panel Note 675	Panel
	Primary Recrystallisation and Void Swelling Behaviour of Cold-Worked, Niobium-Stabilised Austenitic Steels	R. M. Boothby et al	1983	Proc. Conf. on Dimensional Stability and Mechanical behaviour of Irradiated Metals and Alloys, Brighton, Volume 1, pp 5-8	Conference
	Void Swelling in PFR Core Materials	C. Brown et al	1983	Proc. Conf. on Dimensional Stability and Mechanical behaviour of Irradiated Metals and Alloys, Brighton, Volume 1, pp29-31	Conference
	Component Deformation in PFR	J. K. Butler et al	1983	Proc. Conf. on Dimensional Stability and Mechanical behaviour of Irradiated Metals and Alloys, Brighton, Volume 1, pp 203-206	Conference
DFMC/P(83)7	A Review of the Void Swelling of Core Components Irradiated in PFR	G. A. B. Linekar et al			Conference
DFMC/P(84)29	The Density Determination of Specimens from Sub-Assembly Wrapper JTS and Guide-Tube Wrapper VHX Measured Using the	M. Nicholas			
DFMC/P(84)28	The Density Determination of Specimens from Sub-Assembly Wrapper GDG	M. Nicholas			
ND Memo 1739(D)	The Dimensional and Density Changes Produced in 20% CW EN58B Wrapper Material by Thermal Aging	R. M. Sharpe, D. K. Appleby			Memo
	Void and Recrystallisation behaviour of CW En58B Irradiated in UK-1 Experiment in EBR-2	C. Brown et al	1984	PFR Cladding Panel Note 776	Panel
DFMC/P(83)23	Sub-Assembly Corner Post Distortion – Further Examination of DWJ and WLT	B. E. Hart, R. R. Higginson			
DFMC/P(83)8	Progress Report on Grid Leg Distortion	B. E. Hart			
ND Memo 2933(D)	Grid Leg Distortion in PFR Sub-Assemblies	C. Brown et al			Memo
	Void Swelling in Titanium- Stabilised Type 316 Stainless Steel	J. I. Bramman and T. M Williams	1979	PFR Cladding Panel Note 500	Panel
	The Effect of Silicon and Titanium on Void Swelling and Phase Stability in 12Cr/15Ni/Fe Alloys Irradiated with 46 MeV Nickel	D. J. Mazy et al	1979	Proc. Int.Conf. on Irradiation Behaviour of Metallic Materials for Fast Reactor Core Components, Ajaccio, pp61-67	Conference
	Fabrication History of Commercial Cast of P316	L. Williamson	1978	PFR Cladding Panel Note 481	Panel
	Void Swelling Simulation Data on UK P316 Austenitic Alloys	D. J. Mazy et al	1983	PFR Cladding Panel Note 752	Panel
	Initial Swelling Results from Cylinders of M316, P316, FV548, PE16, FI, FV448, FV607 Irradiated between 400 and 650°C to 2J	S. Watkins	1980	PFR Cladding Panel Note 604	Panel
ND Memo 2979 (R)	Neutron-Induced Void Swelling Data on Solid Cylinders Irradiated in UK-1 Experiment in EBR-2 (Data up to 75 dpa NRT Fe)	J. S. Watkins	1980		
	The Effect of Thermal and Mechanical Treatment on the Microstructure of a Precipitation Hardening Alloy – Thesis submitted f	G. A. B. Linekar	1979		
	The Strength and Ductility of Nitrided 20%Cr 25%Ni Steels Containing Titanium in Proc. The Microstructure and design Alloys	H. E. Evans	1973	Cambridge, pp190-194	
	Void Spatial Regularity in an Electron Irradiated Stainless Steel	S. B. Fisher, K. R. Williams	1977	Radiation Effects, Vol 32, pp 123-124	
	Void Studies of Stainless Steel Containing Silicon and Titanium	P. K. Madden, F. B. Fisher	1982	Physica Status Solidi (a), Vol 69, pp569-588	
	Void Swelling in Ion-Irradiated Austenitics Fe-Cr-25Ni Alloys	D. J. Mazy et al	1983	Proc. Conf. on Dimensional Stability and Mechanical behaviour of Irradiated Metals and Alloys, Brighton, Volume 1, pp 9-12	Conference
TRG Memo 692(D)	First Impressions of the Void Swelling and Precipitation Effects Observed in TiN Strengthened 20wt%Cr-25wt%Ni Austenitic S	R. B. Jones et al	1984	PFR Cladding Panel 813	Panel
	Post Irradiation TEM and Density Examination of a High Phosphorus Stainless Steel	C. Cawthorne	1975		Memo
	Void Formation in Quenched Steel	A. F. Rowcliffe	1971	Proc of Conf. on Radiation Induced Voids Metals, Albany, pp 605-611	Conference
	The Effect of Phosphorus on the Swelling and Precipitation Behaviour of Austenitic Stainless Steels during Irradiation	E. H. Lee	1984	J Nucl. Mat., Vol. 122 and 123, pp 299 – 304	
	Proposals for evaluating the suitability of High Phosphorus Austenitic Steels for Use as a Fast Reactor Core Structural Material	C. Cawthorne, M. Stevens	1982	PFR Cladding Panel Note 706	Panel
	Mixed Oxide Fuel Performance	E. Edmonds et al	1979	Topical Meeting Proceedings – International Conference on Fast Breeder Reactor Fuel Performance, Monterey, p54 – 63	Conference
ND-Memo-2934(D)	Differential Void Swelling between BPD Pipe and Guide Tube Wrapper – Destructive Examination of Guide Tube VHX	C. Brown et al			Memo
DFMC/P(83)25	Immersion Density Changes in STA Nimonic PE16 Specimens Irradiated in Materials Surveillance Programme during Runs 2	R. M. Sharpe et al			
ND-Memo-423(S)	The Detailed Loading Schedules for the UK-1 Void Swelling Experiment in EBR-2	J. S. Watkins	1978		Memo
	Voids Produced in Mild Steel by 1MeV Electron Irradiation	E. A. Little	1972	Radiation Effects, Vol 16, pp 135	
	Dispersion-Strengthened Ferritic Steels as Fast Reactor Structural Materials	J. J. Huet and V. Leroy	1972	Nucl Technology, Vol 24, pp 216	
AERE-Report- 8066	The Effect of Heat Treatment on the Structure and Properties of 12% Cr Steels for Fast Reactor Applications	E. A. Little et al	1975		Report
	Void Swelling in FV607 Ferritic Steel – Irradiation with 1 MeV Electrons	D. R. Arkell, T. M. Williams	1978	J. Nucl.Mat. Vol 74, pp 144-150	
	Void Swelling in Irons and Ferritic Steels	E. A. Little, D. A. Stow	1979	J. Nucl. Mat.,Vol 87, pp 25-39	
	The Occurrence of an Ordered fcc Phase in Neutron Irradiated M316 Stainless Steel	C. Cawthorne, C. Brown	1977	J. Nucl. Mat. Vol 66, pp 201-202	
AERE- Report 11581	Precipitation in Irradiated and Unirradiated Austenitic Steels	T. M. Williams	1982	Proc. Conf on Effects of Radiation on Materials, Scottsdale, ASTM STP 782, pp 166 – 185	Conference
	Precipitation in Neutron Irradiated Type 316 Austenitic Steel	T. M. Williams	1984	Proc Conf on Stainless Steel, Gothenberg	Report
	A Study of Void Formation in Fast neutron Irradiated Metals	S. D. Harkness, Che-Yu. Li	1971	Met. Trans. Vol. 2, pp 1457 – 1470	
	The Rate Theory of Swelling due to Void Growth in Irradiated Metals	A. D. Brailsford, R. Bullough	1972	J. Nucl. Mat., Vol 44, pp121-135	
	A Microstructural Explanation for the Low Swelling of Ferritic Steels	R. Bullough et al	1980	Proc Conf. on Effects of Radiation on Materials, Savannah, ASTM STP 725 pp 593-609	Conference
AERE-Report-11525	The role of Void Nucleation in the Swelling Resistance of Ferritic Steel	E. A. Little	1984		Report
	Recent Experimental and Theoretical Insights on the Swelling of Austenitic Alloys	F. A. Garner and W. G. Wolfer	1983	Proc. Conf. on Dimensional Stability and Mechanical Behaviour of Irradiated Metals and Alloys, Brighton, Vol. 2, pp21-24	Conference
ND-Memo-2053 (D)	Void Swelling in 20% CW M316 Cladding from the DFR Mk2 and 2B Sub-assembly Pins	C. Brown, E. J. Fulton	1982		Memo
	Proc. Int. Conf.on Materials for Nuclear Reactor Core Applications, Bristol	BNES with European Nuclear Society an	1987		Conference
	Proc. Int Conf. on Fast Reactor Core and Fuel Structural Behaviour, Inverness	BNES	1990		Conference
	Distortions and Interactions in a Fast Reactor Core, from Mechanics of Solids	J. F. W. Bishop (Edited by H.G.Hopkins and M.J. St		The Rodney Hill 60th Anniversary Volume	
CFWG/P(93)25	A Bibliography of Reports relating to TRAFIC Fuel Pin Performance Code	D. G. Martin			
FPSG(91)P5	Fast Reactor Modelling in Retrospect	J. R. Matthews			
	The calculation of Stress in Fast Reactor Fuel Elements	J. R. Matthews	1970	Proc Int meeting on Fast Reactor Fuel and Fuel Elements, ANS, Karlsruhe	Meeting
	On the Numerical Calculation of the Behaviour of Fuel Elements	M. R. Hayns, D. Wilmore	1973	Proc Conf on Physical Metallurgy of Reactor Fuel Elements, Berkeley	Conference
AERE-R 6586	FRUMP – A Physically Based Fuel Model	D. Wilmore, J. R. Matthews	1979	Proc Conf on Fast Breeder Reactor Fuel, Monterey, California	Conference
AERE-R 6792	Thermal Stresses and Cracking in Fast Reactor Fuel Pellets	J. R. Matthews	1971		
AERE-R 6014	An extension of Anderson's Model for the Axial Extrusion of a Swelling Fuel	J. R. Matthews	1971		
AERE-R7380	An Approach to Nuclear Fuel Modelling	J. R. Matthews	1969		
	FRUMP – a Computer Programme for the Calculation of Stresses in Reactor Fuel Pins	M. R. Hayns, D. Wilmore	1973		
	Paper D1/8	M. R. Hayns, D. Wilmore	1973	Proc 2nd Int Conf on Structural Mechanics in Reactor Technology, Berlin	Conference
TRG Memo 7262 (S)	SLEUTH Fuel Pin Performance Calculations for the Power Cycling Experiment DFR 50	R. L. Moss	1976		Memo
DFR 235/22, TRG Memo 7218 (S)	SLEUTH Fuel Pin Performance Calculations for the Trefoil Experiment	R. L. Moss	1976		Memo
ND-M 116 (S)	SLEUTH Fuel Pin Performance Calculations for the MTR Power Cycling Experiment 9	R. L. Moss	1978		
TRG Memo 7358 (S)	SLEUTH Performance Calculations for the American Fast Reactor Pin FOB	E. P. Hicks	1975		Memo
AERE-R 8010	Computer Model Calculations of Fuel Pin Behaviour for Projected PFR Start-up Conditions	M. R. Hayns	1975		
AERE-R 9098	The Initial Stages of Vibro Fuel Sintering	J. R. Matthews	1978		
AERE-R 11002		RD. G. Martin	1983		

Reference	Title	Authors	Year	Location	Document Type
	The Thermal Conductivity of Powder beds, a model, some measurements on UO ₂ vibro-compacted micro-spheres, and their α	R. O. A. Hall, D. G. Martin	1981	J Nucl Mat 101	
	The Movement of Lenticular Pores in UO ₂ Nuclear Fuel Elements	C. F. Clement	1977	Nucl Mat 68, 63	
	The Movement of Lenticular Pores in Mixed Oxide (U,Pu)O ₂ Nuclear Fuel Elements	C. F. Clement and M. W. Finnis	1978	J Nucl Mat 75, 115	
TP 710	Analytic Solution to Mass Transport Equations for Cylindrical Nuclear Fuel Elements	C. F. Clement	1977	J Nucl Mat 68, 54	
		C. F. Clement	1971		
	Solid State Transport as a Mechanism of Oxygen Thermo-migration in (U,Pu)O _{2+x}	D. I. R. Norris		J Nucl Mat 79, 118	
AERE - R -10818 (Rev)	The Basis of the TRAFIC fuel Performance Code	J. R. Matthews	1984		
	Chemical Modelling in the TRAFIC Code	M. A. Mignanelli and D. G. Martin	1993	J Nucl Mat 204, 173-179	
	Aspects of Chemical Modelling of Fast Reactor Fuel	M. A. Mignanelli and A. W. Gardner	1987	Nucl Eng Design - 6th Int Seminar on Mathematical/Mechanical Modelling of Nuclear Fuel Elements, Switzerland	Seminar
(SOLGASMIX Code ref)		G. Eriksson	1975	Chemica Scripta 8, 100	
AERE-R 3549	TRAFIC - a Description of Common Blocks and Subroutine Argument Lists	A. R. Laney and R. Thetford	1986		
AEA-InTec 0085	RAFIC2.2: Definition of Subroutines	R. Thetford, P. L. Edwards and I. J. Ford	1990		
AEA-TSD-0459	TRAFIC Modelling Studies of Fast Reactor Fuel Pins during Irradiation	D. G. Martin and M. A. Mignanelli	1995		
	A Re-appraisal of the Thermal Conductivity of UO ₂ and Mixed U,Pu Oxide fuels	D. G. Martin	1982	J Nucl Mat 110, 73	
AERE-R 9686	Recommendations for the Calculation of the Temperature drop Across the Fuel Clad Interface of Fast Reactor Fuel Pins	D. G. Martin	1980		
TP 1262	Segregation in Oxide fast Reactor Fuel	R. Thetford	1987		
AERE-R 11453	A Simplified Model for the Treatment of Molten Fuel by the TRAFIC code	R. Thetford	1984		
AERE-R 12484	PELMEL and GELMEL Two Computer Codes for Calculating the Power to Initiate melting in Fast Reactor Fuel Pins at Start of LI	D. G. Martin	1987		
AERE-M 3522	TRAFIC Calculations on Plutonium redistribution in DFR A1091 and PFR RGC fuel pins	R. Thetford	1985		
CFWG/FPSG/P(89)25, TP1335	Modelling the Onset of Fuel Melting in Fast Reactors	D. G. Martin and J. R. Matthews	1990	Nucl Energy 29, 149	
FEWP/P(88)18	The Effect of Burn-up on Power-to-Melt in EFR Fuel	J. R. Matthews, Y. Yates			
FEWP/P(88)22	Validation of TRAFIC Power-to-Melt Calculations	J. R. Matthews, Y. Yates			
FEWP/P(88)30	Operating Margins for Power-to-Melt in 8.5mm EFR Fuel Rods	J. R. Matthews			
FEWP/P(88)54	Parameters Affecting Power-to-Melt in EFR Fuel	J. R. Matthews			
AERE-M-3576	TRAFIC Pre-calculations of the FACTO-1 Experiment	J. R. Matthews, D. G. Martin	1986		
TP1236	On the Failure of Fast Reactor Fuel Pins	J. R. Matthews, T. Preusser	1985	Also in Proceedings of Conference Harwell 1985	
CFWG/FPSG(91)29	Strategy for the Development of the ENPANNE Code	P. L. Edwards, M. A. Mignanelli			
Report No 05271/T4/016	The Chemical Composition of Fuel/Fission Products in a Failed Mixed (U,Pu) Oxide Fuel Pir	M. A. Mignanelli		PNC Annular Pelleted Fuel Consultancy	Report